



Current Status of KAPP-3&4

KAPP-3

- First **Biennial Shutdown** (since Sept 18, 2024) is nearing completion
- Operated **524 FPDs**, 2.11 HOYs (as on Oct 2024)
- First **Synchronization**: Jan 10, 2021
- First **Criticality**: Jul 22, 2020

KAPP-4

- Operating at **~95%FP**
- Operated **155 FPDs**, 0.62 HOYs (as on Oct 2024)
- First **Synchronization**: Feb 20, 2024
- First **Criticality**: Dec 17, 2023

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Key design features

Standard design is based on 540 MWe TAPS-3&4, except some salient changes:

- PHT main circuit: **2 identical loops** with feeders in **interleaving** geometry.
- **Partial boiling** of the coolant at the coolant channel exit
- Salient changes due to partial boiling:
 - Higher **PHT temperatures** at RIH & ROH
 - Core **flow** marginally increased
 - **Pressurizer volume** increased
 - Two ROHs of the same loops are **interconnected** to avoid flow instability
 - **Feeder sizing** optimized to have uniform quality at channel exit
 - **SG** design is modified with increased **drum height**

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Key design features

- Reactor control using
 - Ionization chambers at low power
 - Self Powered Neutron Detectors (SPNDs) in the low power range
 - Both in the intermediate range
- Bulk power estimates are corrected using selected channel temperature and flow measurements made on the primary side in pre-boiling regime. Above this the secondary side measurements are used to verify the thermal output of the core.
- Regional Overpower Protection Systems: Protection against localized overpowers. Reactor bulk power high trip parameter is replaced by Regional Overpower Protection (ROP) trip parameter.



Key design features

- Bulk power regulation and flux tilt control is by modulating the water level in LZC.
- CTM setback will be available pre-boiling. Reactor setback based on bundle power high and spatial power tilt are also available.
- 12 fuel bundles per channel
- Air cooled Diesel Generator: Emergency power supply during extended SBO
- Primary Containment is lined with Carbon Steel liner to reduce the leak rate.
- Design Life of the plant is considered to be 40 years, subject to review at appropriate stage.
- FOAKs...



Regulatory History of KAPP-3&4

- **Site Evaluation Report** was submitted in Feb 2006
 - Site Evaluation Committee was constituted
- **Design Basis Information** was first submitted to AERB in May 2006
- KAPP-3&4 **Preliminary Safety Analysis Report** PSAR (Sec 1, 2, 3) were first submitted to AERB in Jun 2006
 - Subsequently, other sections were submitted in stages
- **PDSC (KAPP-3&4)** was constituted in May 2006. Scope was extended to **RAPP-7&8** in Dec 2009. Scope was further extended to **GHAVP-1&2** in Apr 2015: Held 90 meetings
 - Just before the start of Commissioning of KAPP-3&4, the PDSC (KAPP-3&4, RAPP-7&8 and GHAVP-1&2) was reconstituted as **PDSC-PHWR**, responsible for design safety reviews of all 700 MWe PHWRs: Held 84 meetings till date
- Based on the Commissioning experience the **Final Safety Assessment Report** (FSAR) of KAPP-3&4 was submitted recently in Nov 2024.

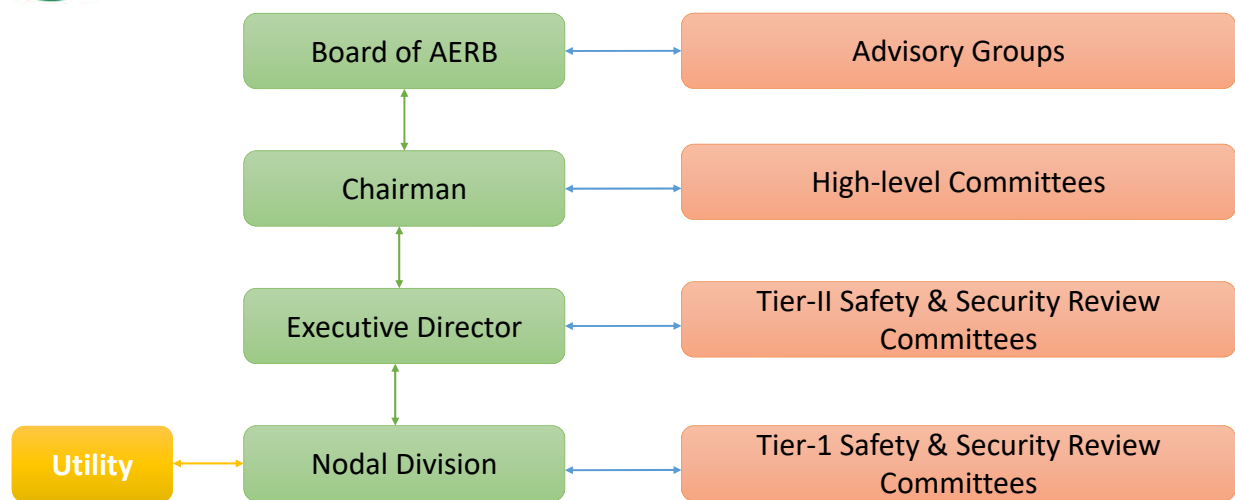
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Multi-tier Review Process of AERB



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Working groups / Specialist groups

- WG-1 (Radiation Shielding)
- WG-2 (Mechanical Design)
- WG-3 (Moderator System)
- WG-4 (PHT System)
- WG-5 (Fuel & FHS)
- WG-6 (Process Water & Rx Aux)
- WG-7 (Control & Instrumentation)
- WG-8 (Electrical System)
- WG-9 (Ventilation & ESFs)
- WG-10 (Reactor Physics)
- WG-11 (Radiation prot & waste mgmt.)
- WG-12 (QA, PSI/ISI)
- WG-13 (Containment Analysis)
- WG-14 (TG & Auxiliaries)
- WG-15 (Accident Analysis)
- WG-16 (Fire Hazard Analysis)
- WG-17 (IV&V)
- WG-18 (Commissioning)
- WG-19 (Technical Specifications)
- TFs: Layout, Common MCR, EQ, Safety Classification/ seismic categorization, Trip coverage, Drop load, Containment test, EPP, 4x100%DG
- CESC & its WGs

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Consents for NPPs

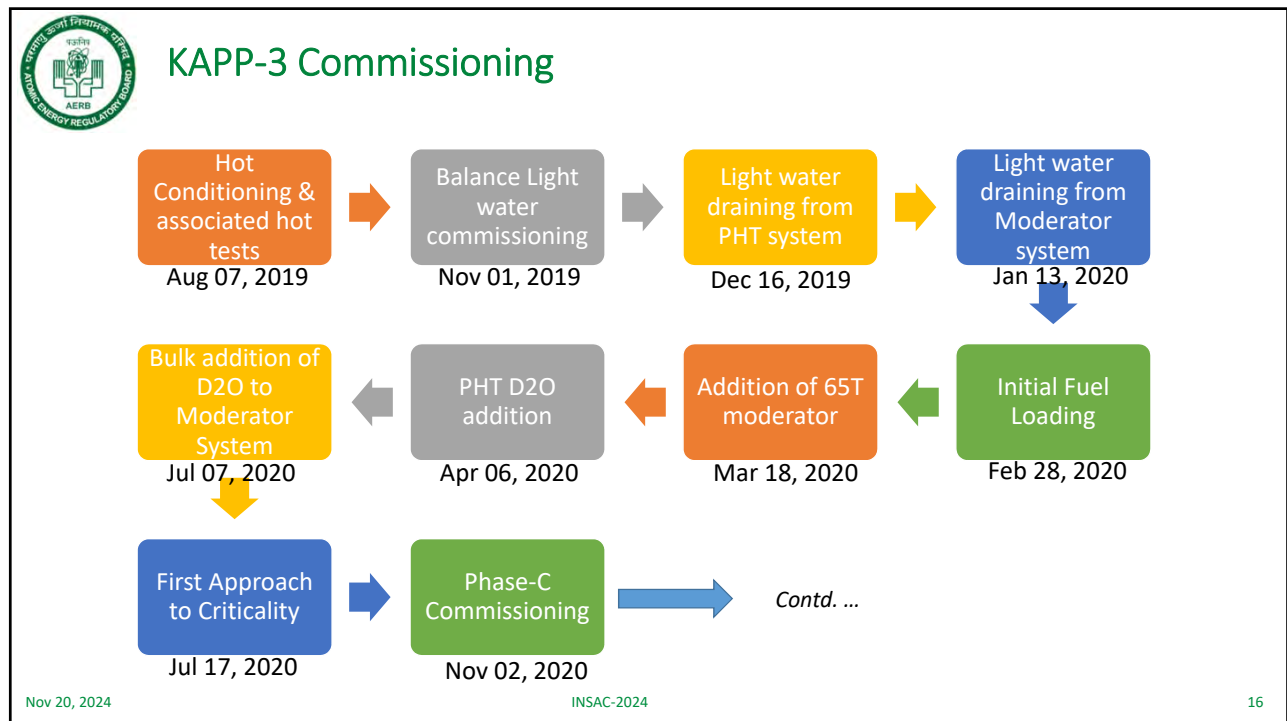
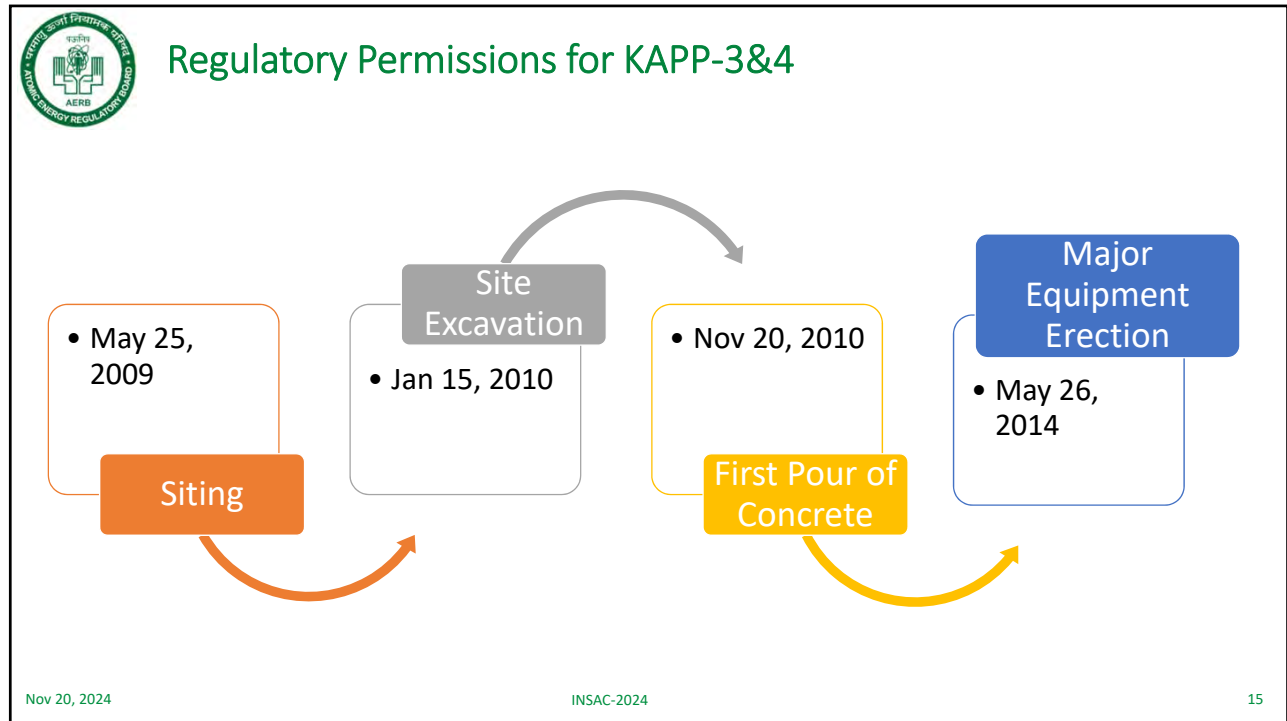
Stage	Activity/ Sub-stages
Siting	Location of NPP at a proposed Site
Construction	Site Excavation
	First Pour of Concrete
	Major Equipment Erection
Commissioning	Phase-A (without fuel)
	Phase-B (with fuel)
	Phase-C (power raise)
Operation	Periodic Safety Reviews

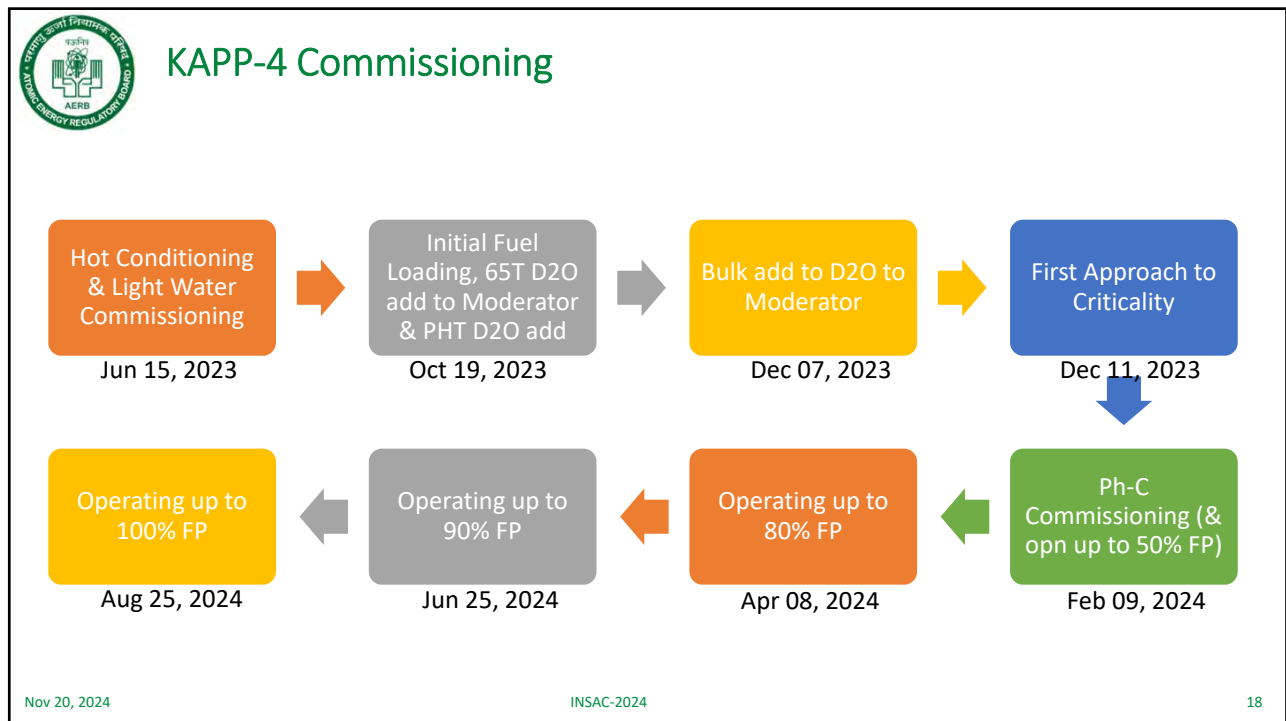
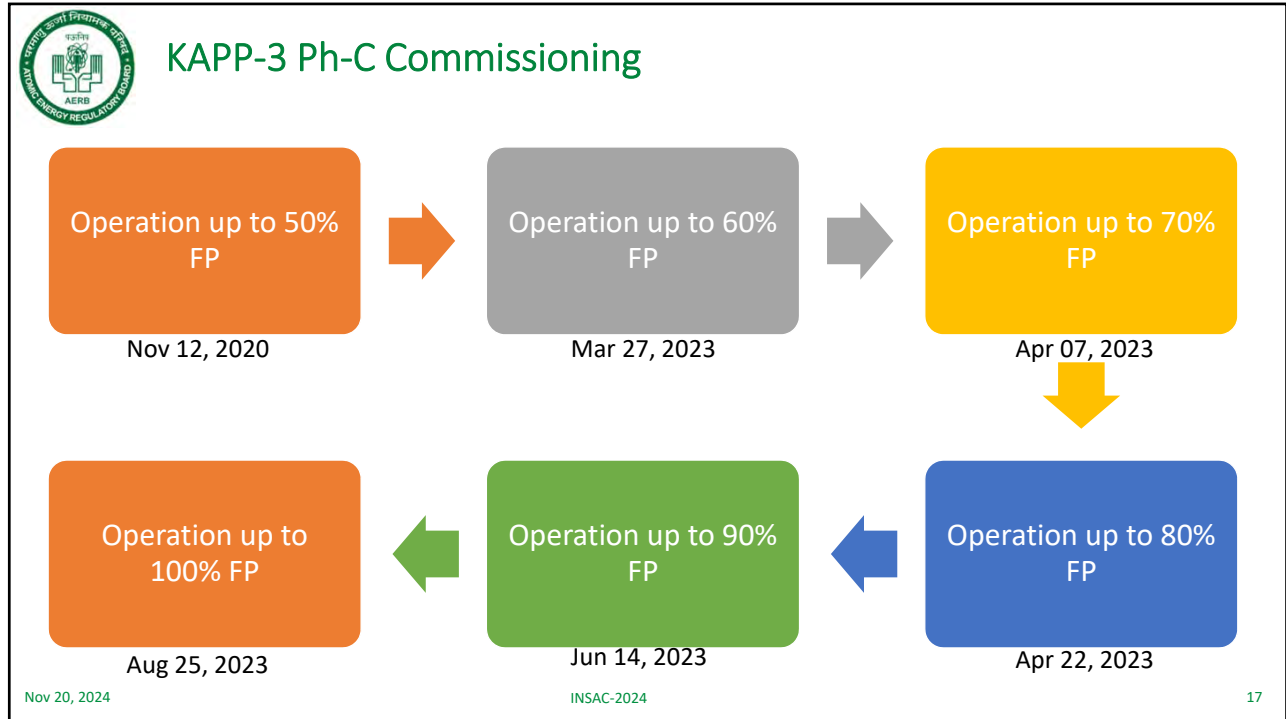


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Key Challenges

- **First of a kind** systems / features
- **Issues** in Siting, Design, Construction, Commissioning
- New / Revised **Regulatory Documents**: AERB/SC/D (2009, 2014), SC/S (2014), SG/D-19 (2018), SM/RIA (2019), SG/O-2 (2024), Under review SC/G, SG/G-1, SC/QA, SG/QA
- Change of guard (2010-2024):
 - At NPCIL HQ & Site
 - At AERB
- Advent of AERB's Integrated Management System (2015)
- AERB's **Safety Review & Licensing process**: Reliance on Committees → Strengthening in-house competencies
 - Maximum in-house Review Groups
 - Committees Recommendatory in nature
- Regulatory **Inspection**: Focus on issues → Focus on Systemic deficiencies
- **COVID**: Commissioning of KAPP-3

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FOAK systems / features in 700 MWe PHWRs

- **PHT Partial boiling**
- **Feeder Interleaving**
- **Primary Containment Liner**
- **Passive Decay Heat Removal System**
- **Containment Spray System**
- **Super Heavy Concrete**
- **Mobile Transfer Machine**
- **Regional Overpower Protection System**
- **Containment Filtered Venting System**
- **Passive Catalytic Re-combiner Devices**

Followed rigorous reviews of experiments, analyses, Literature, mock ups and Commissioning tests.

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PHT Partial Boiling

- Partial boiling at coolant channel outlet to extract 19 % more power from 540 MWe core
- ROH inter-connection of same loop to avoid any flow instability
- Higher PRZ volume to accommodate swelling
- Rx Thermal Power measurement from secondary side parameters
- Periodic lowering of Rx power to pre-boiling regime to detect channel flow blockage
- Size of feeders increased
- End shield thickness reduced

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PHT feeder Interleaving

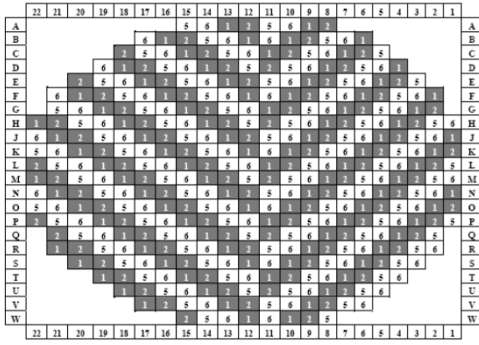
- Reactor headers length is increased to distribute feeders throughout the Core.
- Interleaved configuration has higher safety margins
 - Reduction in peak power (Void gets distributed across the core during LOCA)
 - Increase in Prompt Criticality Margin
 - Reduction in adiabatic heat deposition on Fuel pins
 - Lower sheath temperature and lower clad oxidation

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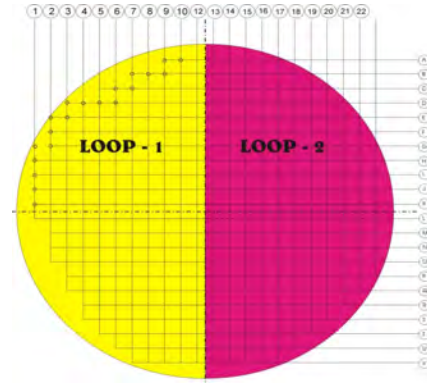
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Interleaved core of 700 MWe Design



TAPS-3&4 Core Configuration



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Containment Liner

- PC is provided with 6mm thick CS integral liner
- Pre-fabricated Ring Liner at KAPP-3&4: Minimum dimensional deviations
- Leak rates:
 - Unlined containments: 0.3% v/v per hour
 - KKNPP-1&2: 0.3% v/v per day
 - 700 MWe PHWRs: 1% v/v per day

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Prefabricated ring liner at KAPP-3&4



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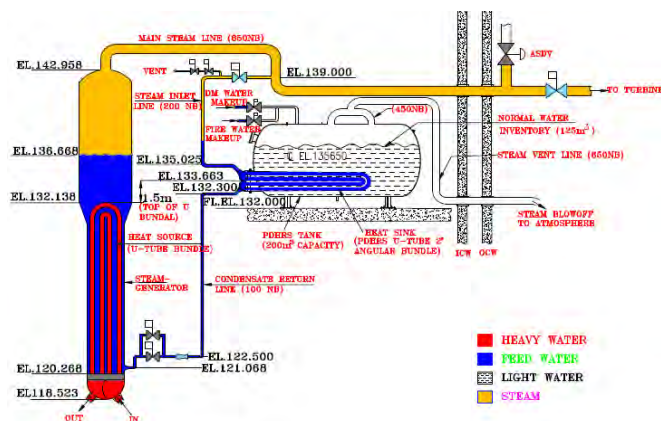
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Passive Decay Heat Removal System (PDHRS)

- Additional safety feature to maintain SG inventory during SBO condition.
- Valves on condensate return line opened remote manually within 15 minutes in case of SBO.
- EC inventory adequate for 8 hours following SBO.
 - Provision for Fire water injection in PDHRS tanks

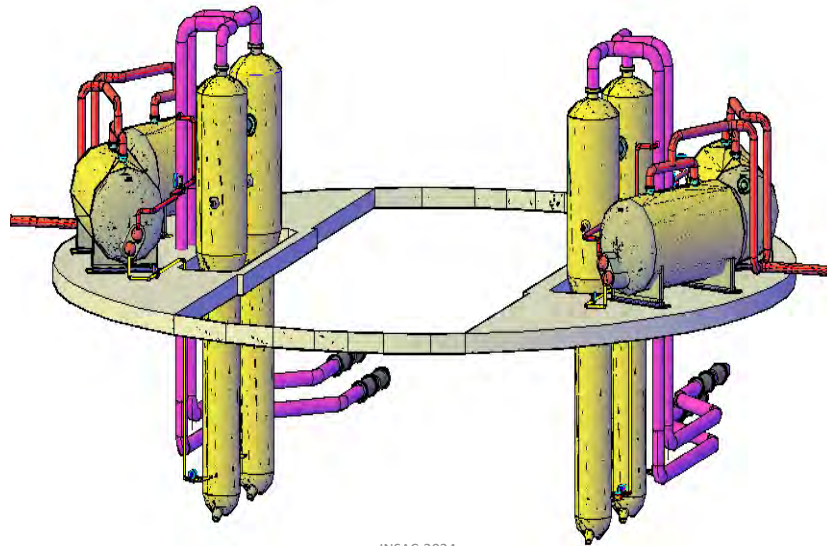


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Schematic layout of PDHRS



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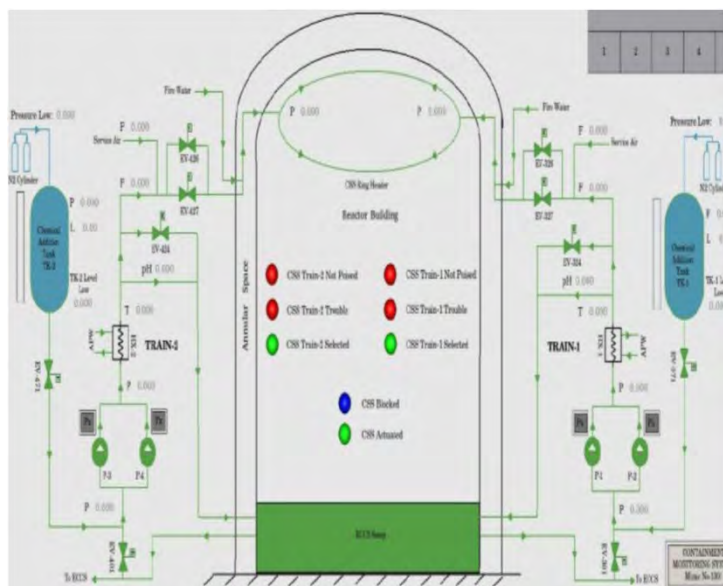


Containment Spray System

For removal of Fission products & containment cooling / depressurization

Substitutes

- PCFPB
- Suppression pool, Distribution Header & Vent Shafts
- RB Coolers (not designed for accident cond.)



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Super Heavy Concrete

- Required for compact layout of 700 MWe NPPs
- Density: 4.2 gm/cc
 - Used 4.1 gm/cc (small quantity) at TAPS-3&4
- Used in following locations of 700 MWe PHWRs:
 - FT Room
 - Access tunnel (pump room)



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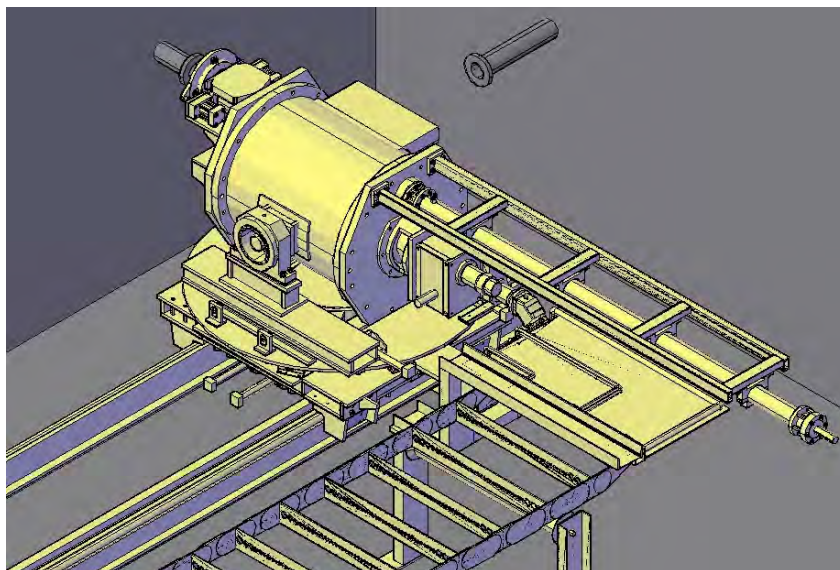
Mobile Transfer Machine

- Improved design for fuel transfer
- Eliminates/ combines the following systems of 220/540 NPPs:
 - Shuttle transfer station
 - Shuttle transport tube
 - Transfer Magazine
- Eliminates D₂O from FT system
- Common Machine for both N/S sides & discharges spent fuel bundles into the TLB.

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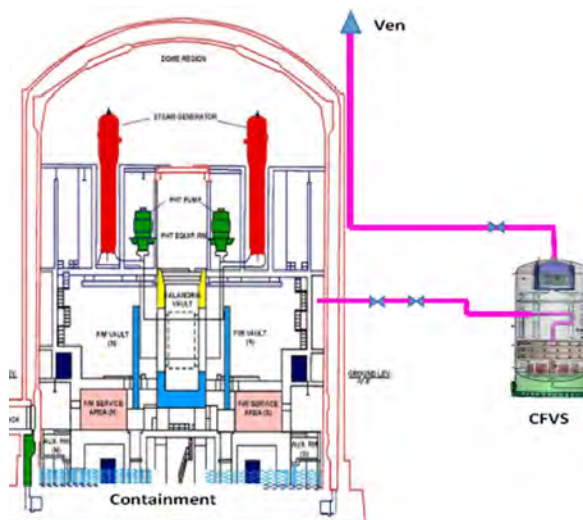
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Containment Filtered Venting System (CFVS)

- Will be operated @1.55kg/cm² containment pressure, after ensuring hydrogen concentration < 4% v/v.
- Decontamination Factor:
 - Particulates and Elemental Iodine = 100
 - Organic Iodine = 10



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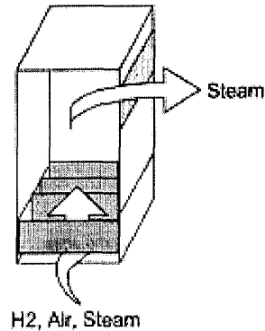
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Passive Catalytic Re-combiner Devices

- In DBA, PCRDs are not required as hydrogen management will be taken care by layout. In BDBA, PCRDs will start working at ~2% hydrogen concentration.



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Post-Fukushima Safety Upgrades

- External hook up points: SG, PHT, ECCS, ES, Calandria, CV, SFSB, CSS, PDHRS, ECCS sump
- Containment Filtered Venting System (CFVS)
- Passive Catalytic Recombiner Devices (PCRD)
- Automatic reactor trip during a seismic event
- Fixed Air Cooled DG sets located in BDBE qualified NFSB
- Mobile driven trailer mounted pumps, fire tenders, fixed submersible pumps at SRPH
- Hydrogen & Steam Concentration Monitoring system (HSCMS)
- Severe Accident Parameter Monitoring System (SAPMS)
- On-Site Emergency Support Centre (OESC)
- Real time Online Decision Support System (RO-DSS)

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Key issues resolved in Siting / Design

- **Three Zone Concept**: Zone 2 → Zone-1 (inactive) & 3 (active)
- **End shield thickness** reduced: Facilitate layout of higher sized feeders & easy replacement of single channel
- Common Main **Control Room**
- **Shutdown cooling pumps** and **HX** brought down from pump room to accessible area and PRC area respectively.
- Shortfall in **Exclusion Zone**, major District Road passing near to Site boundary wall
- Fishing activities in Ponds within Exclusion Zone: **No fishing zone** declared in coordination with Gujarat Government
- Dam break analysis suggested **lowering of roads** in 3 stretches
- Low Trajectory Turbine Missile Zone: CB, SAB in **LTTM** zone of KAPP-3&4, IDCT, SRPH, FWPH in LTTM zone of KAPS-2. Now only once IDCT (out of two loops)

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Key issues resolved in Construction

- Excessive **chipping of End shield support structure** for removal of angles for template used for locating anchor bolts
- Shift of **Calandria vertical reference plane** due to error in transferring Centre line markings to top hatch floor
- **Calandria Vault top hatch** beam manufacturing & QA was inappropriate
- **Moderator HX cover studs** were defective (found during Site PSI, missed during Manufacturing)
- Requalification of **long-procured equipment**
- **Basis of Acceptance** for the safety critical equipment
- Compliance to Factories Act, 1948 and **AER(F)R 1996**
- **ASNT/ISNT** Level-II certification

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Key issues resolved in Commissioning

- Commissioning of **FOAK** systems
- Finalization of **Trip Set points**
- **PSI of Supports** as per ASME Sec XI IWF-2200
- **Temperatures** beyond design in **high enthalpy areas** of Reactor Building
- Consideration of **internal leakages in PHT system** during Station Black out condition
- Issues in **Delayed Neutron Monitoring System SVs** leading to incapacitated failed fuel detection
- **Foreign material** in feeder inlet orifice
- Reduced capacity of **Deaerator Relief Valves**
- Channel temperature monitoring system **RTDs** faulty

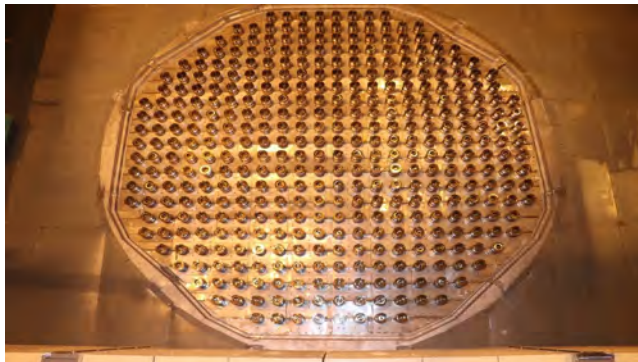
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KAPP-3&4



FM Vault



Turbine

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KAPP-3&4



Main Control Room



Fuel handling control panels



GHAVP-1&2



GHAVP FWPH



GHAVP Geotech investigations



KGS-5&6



KGS-6 NB area



KGS-5&6 Main plant area

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